

ENHANCED Shield

A Critical Materials Technology Enabling Compact Superconducting Tokamaks

GAMOW Kickoff Meeting
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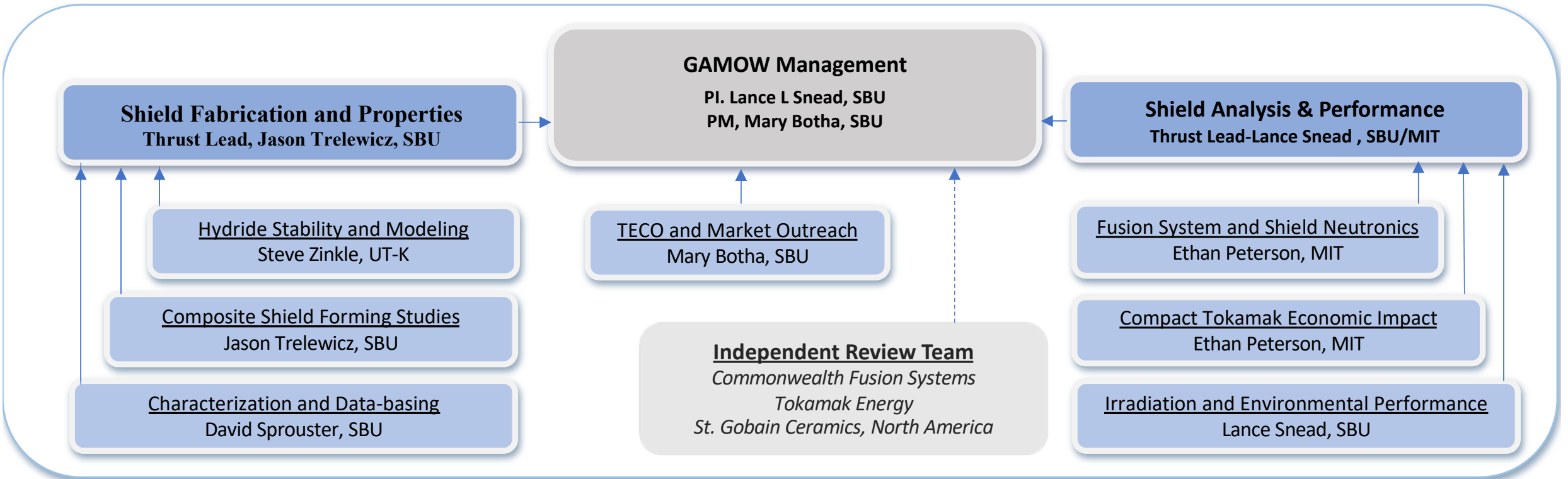
Steve Zinkle, University of Tennessee-Knoxville

Jacopo Buongiorno, Massachusetts Institute of Technology



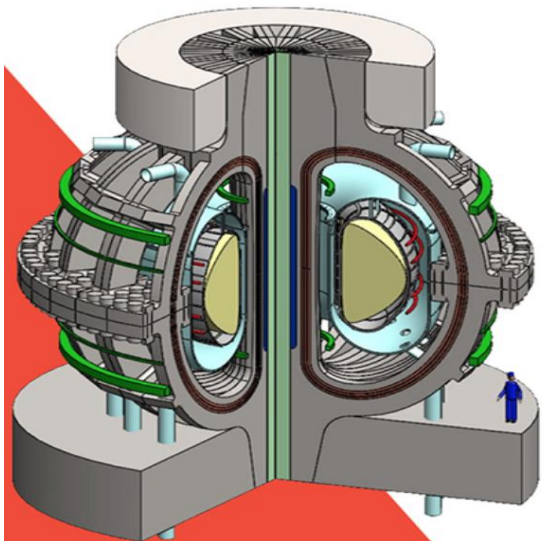
Enhanced Shield Team members and roles

A program of iterative Materials and Fabrication R&D guided by Analysis

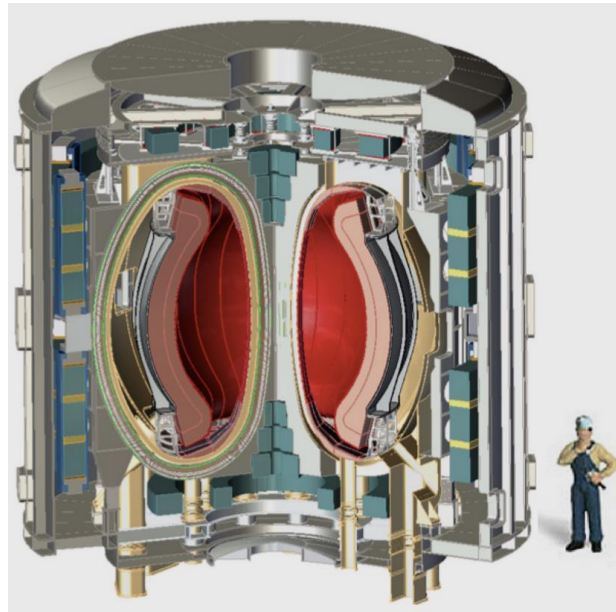


High-level motivation, innovation, and goals of the project

- ▶ Compact high-temperature superconducting fusion reactors offer promise of high performance though are limited by current magnet shield.

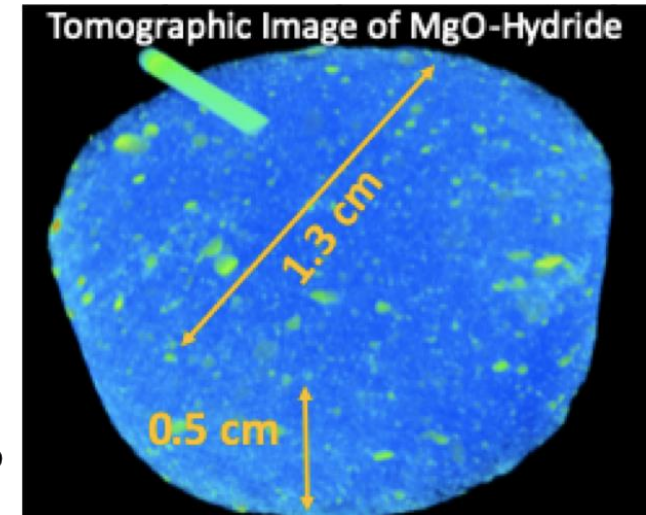
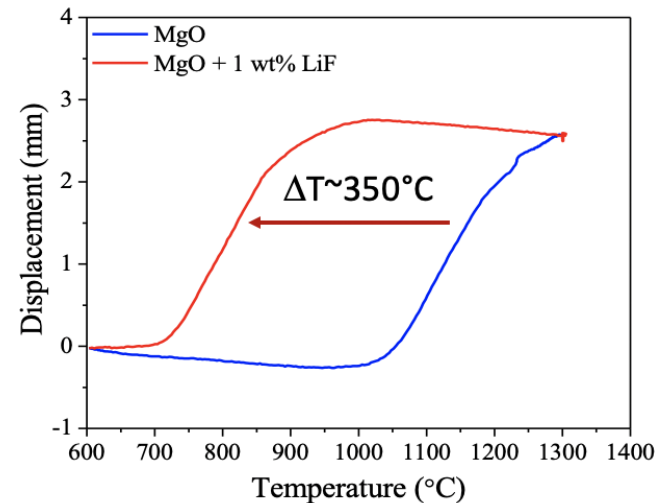


CFS-ARC Reactor



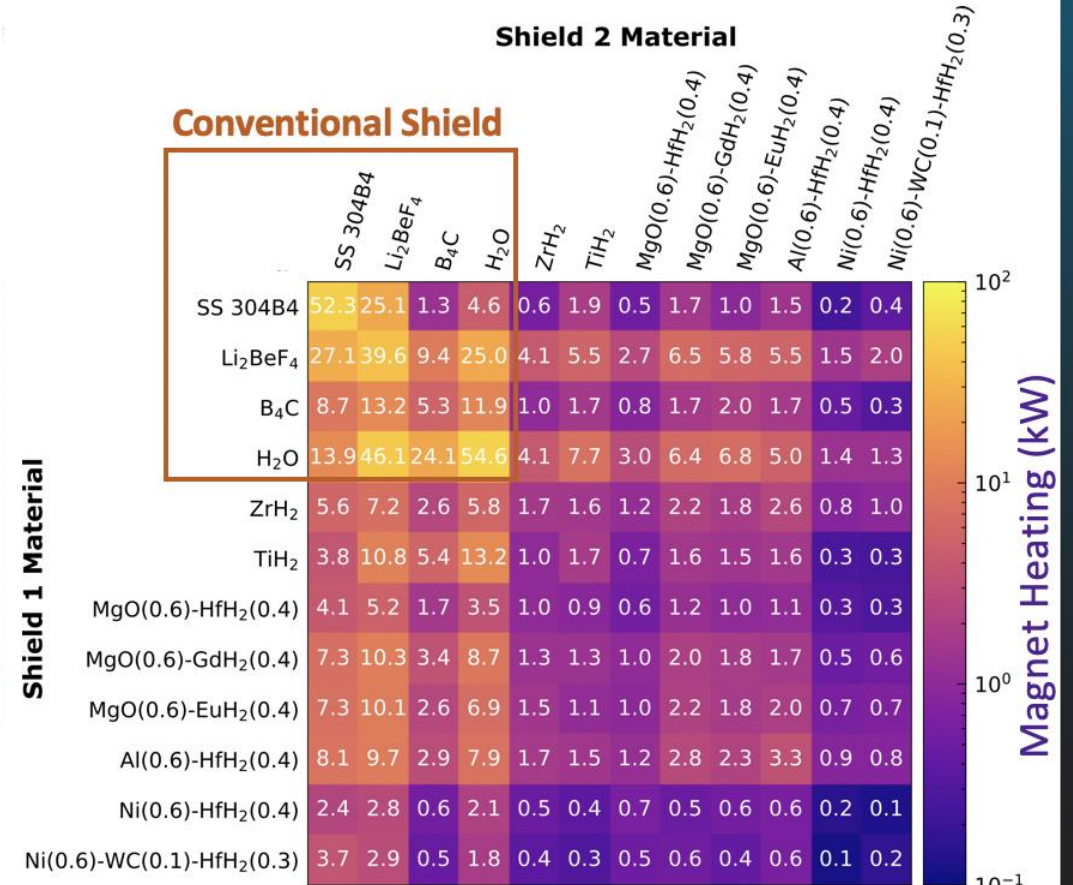
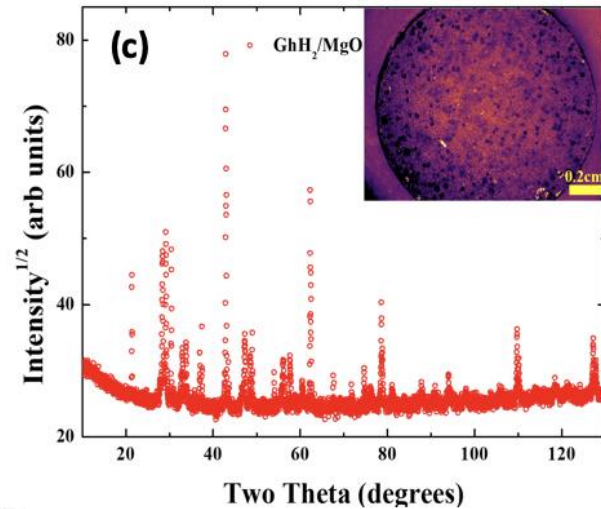
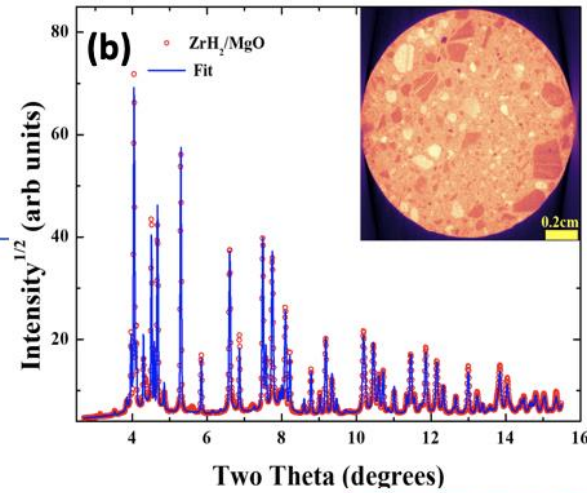
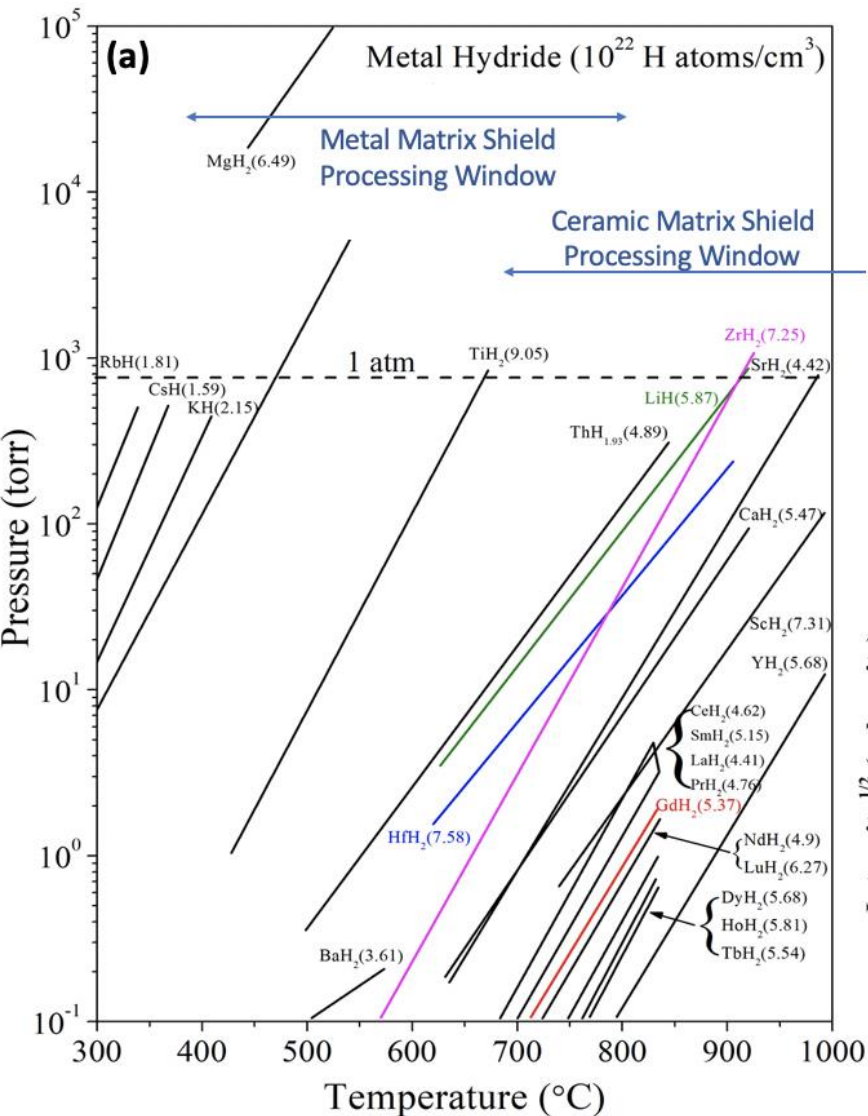
TE ST-40 Reactor

- ▶ While hydrides are attractive neutron shields, safety and H-mobility preclude their use. Here we advance composites that entrains neutron absorbing hydrides in stable ceramic and metal-matrix composite forms.



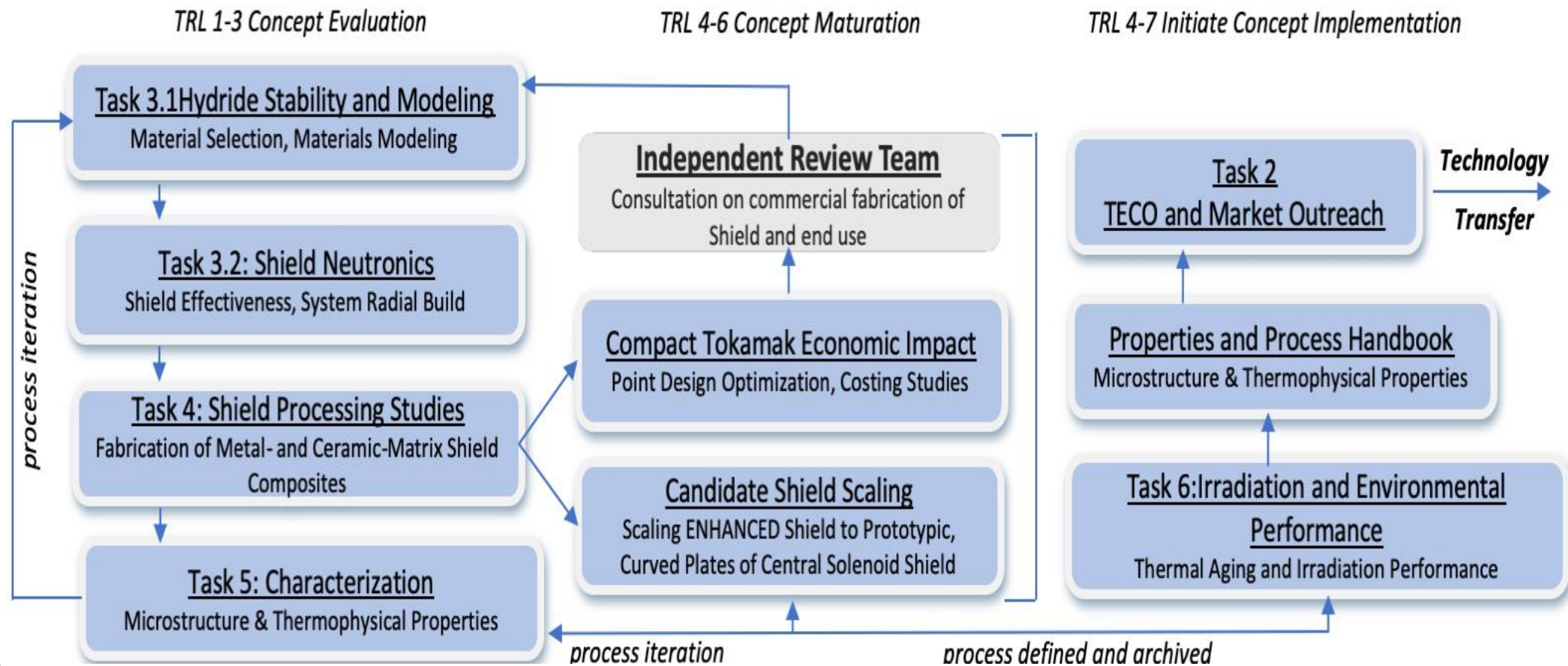
- ▶ Goal of reducing both nuclear heating and material damage (neutron dose) by $\gg 10x$ in a practical composite shield.

High-level motivation, innovation, and goals of the project



Major tasks, milestones, risks, and desired project outcomes

- ▶ Major milestone is the demonstration of a fabricable composite of adequate absorbing metal hydride density
- ▶ Major risks include loss of hydrogen at required processing temperature and under irradiation.



TECO and desired project outcomes

	State of the Art	Reactor	Milestone Metrics*
Technical Metrics			
1: Compact HTS Fusion Reactor Lifetime Inboard Shield	Current designs based on Steel, Boron Compounds, Tungsten bulky and require scheduled replacement.	CFS ARC Reactor TE-ST-40	ENHANCED Shield yielding <math><3E18\text{ n/cm}^2</math> fast in 30 years Heat deposit <math><50\text{ W/m}^3</math> To inboard HTS TF Coil Matrix H/T diffusivity <math><10^{-10}\text{ cm}^2/\text{s}</math> Operating Temp. to 200°C H Loss/Redistribution <math><15\%</math>
2: Compact HTS and Normal Conducting Outboard Shield	Use of Borated Steel, H ₂ O Assumed.	EU Demo HCPB	-As above, with...- Operating Temp. to 600°C Stretch Operating Temp 800°C H Loss/Redistribution <math><20\%</math>
3: Scale Fabrication of Metal and Ceramic Matrix-Neutron Absorbing Hydride Shield	Scaling MgO-ZrH ₂ in ARPA-E MEITNER. No other similar tech.	CFS ARC Reactor Inboard Shield	Curved Plates 10 x10 cm ² Uniform H-content <math><15\%</math> Uniform density & K _{th} , <math><20\%</math>
Cost Metrics			
1: Reduced radial build & overnight construction cost	Assume current SPARC W/Steel/B ₄ C	CFS ARC Reactor TE-ST-40	>25% reduced shield thickness >20% reduced plant capital cost
2: Increased plant availability	n/a		>5% increase

Aspirational follow-on plans

- ▶ Demonstration of Commercial Viability
 - Scale to repeatable large block.
 - Engage commercial fabricator to transfer technology.
 - Expand engagement with reactor designers.
- ▶ Improving understanding and TRL
 - Expanded modeling and performance information under irradiation.
 - Define limits of thermal performance.
 - Complete understanding of effect of hydrogen transport, redistribution, and loss.

Independent Review Team

Commonwealth Fusion Systems

Tokamak Energy

St. Gobain Ceramics, North America